

## INTRODUCTION

- General Fusion (GF) implements a Magnetized Target Fusion (MTF) approach in which a metal liner compresses a compact toroid plasma to fusion conditions.
- To guide the design of GF MTF systems, we need to simulate plasma dynamics inside a domain with moving boundaries. For this purpose, GF has developed the ISM-plasma code as a part of the Integrated System Model (ISM) project.
- The ISM-plasma code is based on assumption that slow resistive evolution of magnetically-confined plasma can be represented as a sequence of 2D axisymmetric Grad-Shafranov equilibria, which are linked through 1D transport processes [1].
- The code uses a mixed Eulerian-Lagrangian contour method: the Grad-Shafranov equilibrium is solved on a fixed mesh (Eulerian description) and transport processes are discretized on a set of contours advected by plasma motion (Lagrangian description).
- The main advantages of the ISM-plasma code are:
  - speed (the numerical time step is limited by a low resistive diffusion timescale and not by a fast Alfvén timescale as in usual MHD codes);
  - compatibility with moving boundaries of the plasma domain (especially useful for modeling MTF compression systems).
- We present several test cases to verify the ISM-plasma code against other MHD codes and demonstrate its capabilities.

## CONTOUR METHOD

### Assumptions

- System is axisymmetric, magnetic field in plasma is:
 
$$\mathbf{B} = \nabla\psi \times \nabla\phi + F\nabla\psi$$
- At any time  $t$ , plasma is in Grad-Shafranov equilibrium (instantaneous force balance):
 
$$\frac{1}{\mu_0}(\nabla \times \mathbf{B}) \times \mathbf{B} - \nabla p = 0 \Rightarrow \Delta^* \psi + FF'_{\psi} + \mu_0 r^2 p'_{\psi} = 0$$
- Toroidal field function  $F(t, \psi)$  and plasma pressure  $p(t, \psi)$  are functions of poloidal flux  $\psi$ .
- Dynamics of magnetic field, plasma density and pressure follow from single-fluid MHD:

$$\frac{\partial \mathbf{B}}{\partial t} = \nabla \times (\mathbf{v} \times \mathbf{B} - \eta \nabla \times \mathbf{B}) \Rightarrow \begin{cases} \frac{\partial \psi}{\partial t} = -\mathbf{v} \cdot \nabla \psi + \eta \Delta^* \psi \\ \frac{\partial F}{\partial t} = r^2 \nabla \cdot \left( -\frac{F}{r^2} \mathbf{v} + \frac{v_{\phi}}{r} \mathbf{B} + \frac{\eta}{r^2} \nabla F \right) \\ \frac{\partial n}{\partial t} = -\nabla \cdot (n\mathbf{v}) \\ \frac{\partial p}{\partial t} = -\mathbf{v} \cdot \nabla p - \frac{5}{3} p \nabla \cdot \mathbf{v} + \frac{2}{3} (q^T + q^{\Omega}) \\ p = nk_B T \end{cases}$$

- Thermal conduction and Ohmic heating are:
 
$$q^T = \nabla \cdot \left( \frac{3}{2} \chi_{\perp} n k_B \nabla T \right)$$

$$q^{\Omega} = \frac{\eta}{\mu_0} \left( \frac{(\Delta^* \psi)^2}{r^2} + \frac{(\nabla F)^2}{r^2} \right)$$
- Thermal conduction is fast along magnetic field lines (parallel thermal diffusivity  $\chi_{\parallel} \rightarrow \infty$ ), therefore plasma temperature  $T(t, \psi)$  and density  $n(t, \psi)$  are functions of poloidal flux  $\psi$ .
- Plasma magnetic diffusivity (electric resistivity)  $\eta$  and perpendicular thermal diffusivity  $\chi_{\perp}$  are functions of other parameters, we assume they are constant on  $\psi$  contours.
- Problem:** there is no explicit equation for plasma velocity  $\mathbf{v}$ , it has to be determined from conditions that  $F, p, T$  and  $n$  are functions of  $\psi$  at any time; this is not straightforward!
- Solution:** equations in ISM-plasma code are rewritten for passive scalars – quantities advected by plasma flow – poloidal flux  $\psi$ , toroidal flux  $\Phi$  and total number of particles  $N$  inside  $\psi$  contour, and plasma entropy  $s$ ; this removes the need to know  $\mathbf{v}$ :

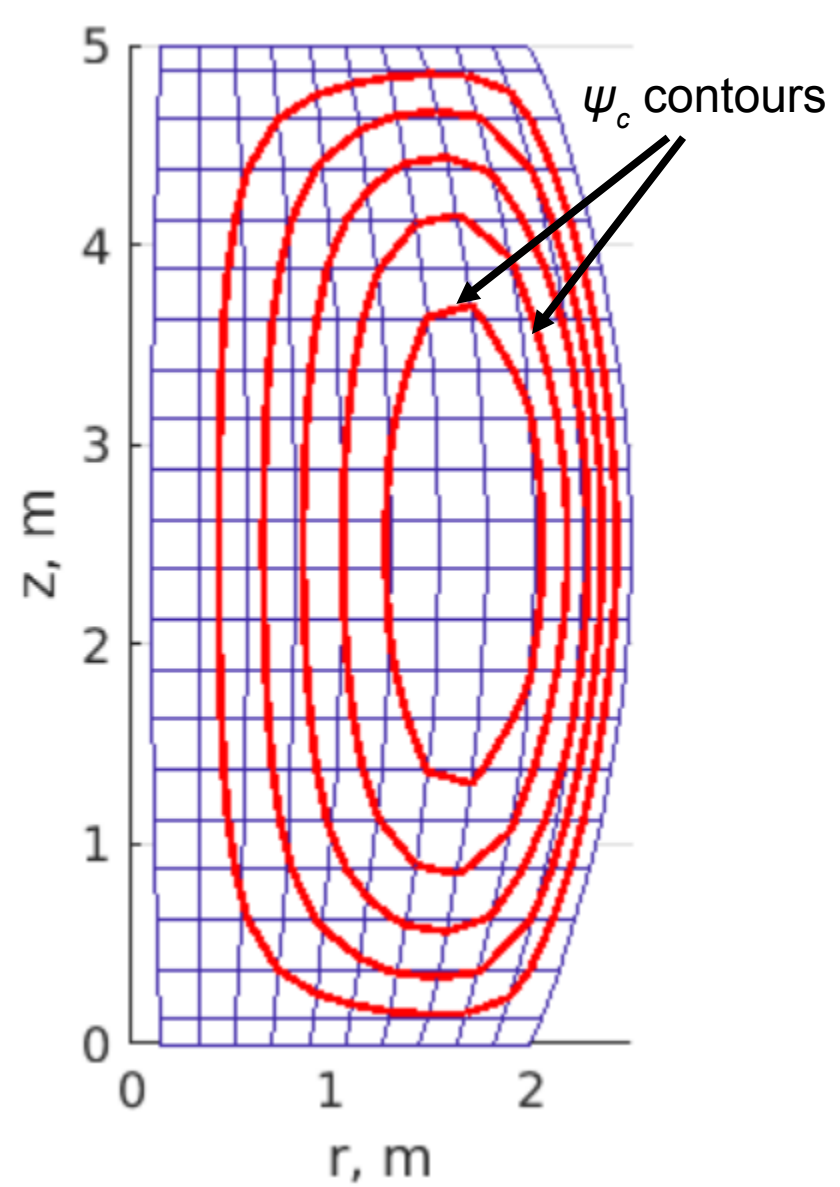
$$\Phi(t, \psi) = \int_{\psi(r,z) > \psi} \frac{F(t, \psi(r, z))}{r} dr dz$$

$$N(t, \psi) = \int_{\psi(r,z) > \psi} n(t, \psi(r, z)) 2\pi r dr dz$$

$$s(t, \psi) = p(t, \psi) n(t, \psi)^{-5/3}$$

### Discretization

- 2D Eulerian mesh is introduced on  $r$ - $z$  plane (indices  $j$ - $k$ ):
  - quadrilateral, structured, conformal to the domain;
  - $z$  mesh coordinates ( $z_k$ , horizontal lines) are fixed;
  - $r$  mesh coordinates ( $r_j = r_j(z_k)$ , vertical lines) change in time if the right boundary (liner surface) is moving.
- Poloidal flux is discretized on this mesh,  $\psi_j = \psi(r_j, z_k)$ .
- 1D functions of  $\psi$  ( $F, p, T, n$ ) are discretized on an array  $\mathbb{E}$  of values  $\psi_c = [\psi_1(\text{axis}), \psi_1, \psi_2, \dots, \psi_c(\text{edge})]$ .
- Levels  $\psi(r, z) = \psi_c$  correspond to a set of contours, these contours form a Lagrangian mesh:
  - they follow the motion of plasma;
  - total number of plasma particles  $N_c = N(t, \psi_c)$  inside each contour  $\psi_c$  remains constant in time;
  - values of  $\psi_c$  change during resistive dynamics.
- Every time step contains two stages:
  - finding new 2D Grad-Shafranov equilibrium;
  - advancing 1D contour-averaged transport equations.



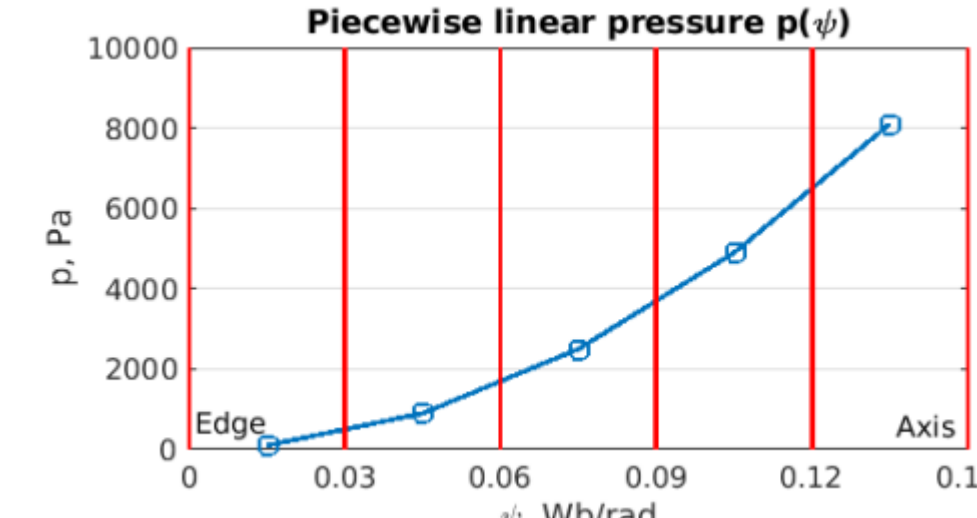
## First stage: 2D Grad-Shafranov equilibrium

- Grad-Shafranov Equation (GSE) is discretized on 2D Eulerian  $r$ - $z$  mesh:
 
$$[\Delta^* \psi + FF'_{\psi} + \mu_0 r^2 p'_{\psi}]_{jk} = 0$$
- At  $t=0$ , functions  $F(\psi)$  and  $p(\psi)$  are given and GSE is solved for 2D poloidal flux  $\psi_j$ .
- At  $t>0$ , functions  $F(t, \psi)$  and  $p(t, \psi)$  are reconstructed from current profiles of toroidal flux  $\Phi(t, \psi)$ , plasma entropy  $s(t, \psi)$  and density  $n(t, \psi)$ :
 
$$F_{c+\frac{1}{2}} = \frac{d\Phi(t, \psi)}{dL(t, \psi)} \Big|_{c+\frac{1}{2}} \approx \frac{\Phi_{c+1} - \Phi_c}{L_{c+1} - L_c}$$

$$n_{c+\frac{1}{2}} = \frac{dN(t, \psi)}{dV(t, \psi)} \Big|_{c+\frac{1}{2}} \approx \frac{N_{c+1} - N_c}{V_{c+1} - V_c}$$

$$p_{c+\frac{1}{2}} = s_{c+\frac{1}{2}} n_{c+\frac{1}{2}}^{5/3}$$
- Inductance and volume inside contour  $\psi_c$ :
 
$$L_c = \int_{\psi(r,z) > \psi_c} \frac{dr dz}{r} \approx \sum_{\psi_{jk} > \psi_c} \frac{\Delta r_{jk} \Delta z_k}{r_{jk}}$$

$$V_c = \int_{\psi(r,z) > \psi_c} 2\pi r dr dz \approx \sum_{\psi_{jk} > \psi_c} 2\pi r_{jk} \Delta r_{jk} \Delta z_k$$
- At the first stage of every time step, GSE is solved for  $\psi_j$  by iterations, keeping fixed values of  $\psi_c, \Phi_c, s_{c+1/2}, N_c$  and updating  $L_c$  and  $V_c$  at each iteration based on new  $\psi_j$ .



## Second stage: 1D transport equations

- At the second stage of every time step, contour-averaged 1D transport equations for poloidal flux  $\psi_c$ , toroidal flux  $\Phi_c$  and plasma entropy  $s_{c+1/2}$  are advanced in time:
 
$$\frac{d\psi_c}{dt} = \eta_c (\Delta^* \psi)_c$$

$$\frac{d\Phi_c}{dt} = Q_c^{\Phi}$$

$$\frac{ds_{c+1/2}}{dt} = \frac{2}{3} n_{c+1/2}^{-5/3} (q_{c+1/2}^T + q_{c+1/2}^{\Omega})$$
- The contour-averaged terms on the right hand side and their discretization:
 
$$\langle \Delta^* \psi \rangle_c = \frac{dL(t, \psi)}{dL(t, \psi)} \Big|_c \approx \frac{L_{c+\frac{1}{2}} - L_{c-\frac{1}{2}}}{L_{c+\frac{1}{2}} - L_{c-\frac{1}{2}}}$$

$$Q_c^{\Phi} = \frac{1}{2\pi} \int_c \frac{\eta}{r^2} \nabla F \cdot d\mathbf{S} = \eta_c I_c F'_{\psi} \Big|_c \approx \eta_c I_c \frac{F_{c+\frac{1}{2}} - F_{c-\frac{1}{2}}}{\psi_{c+\frac{1}{2}} - \psi_{c-\frac{1}{2}}}$$

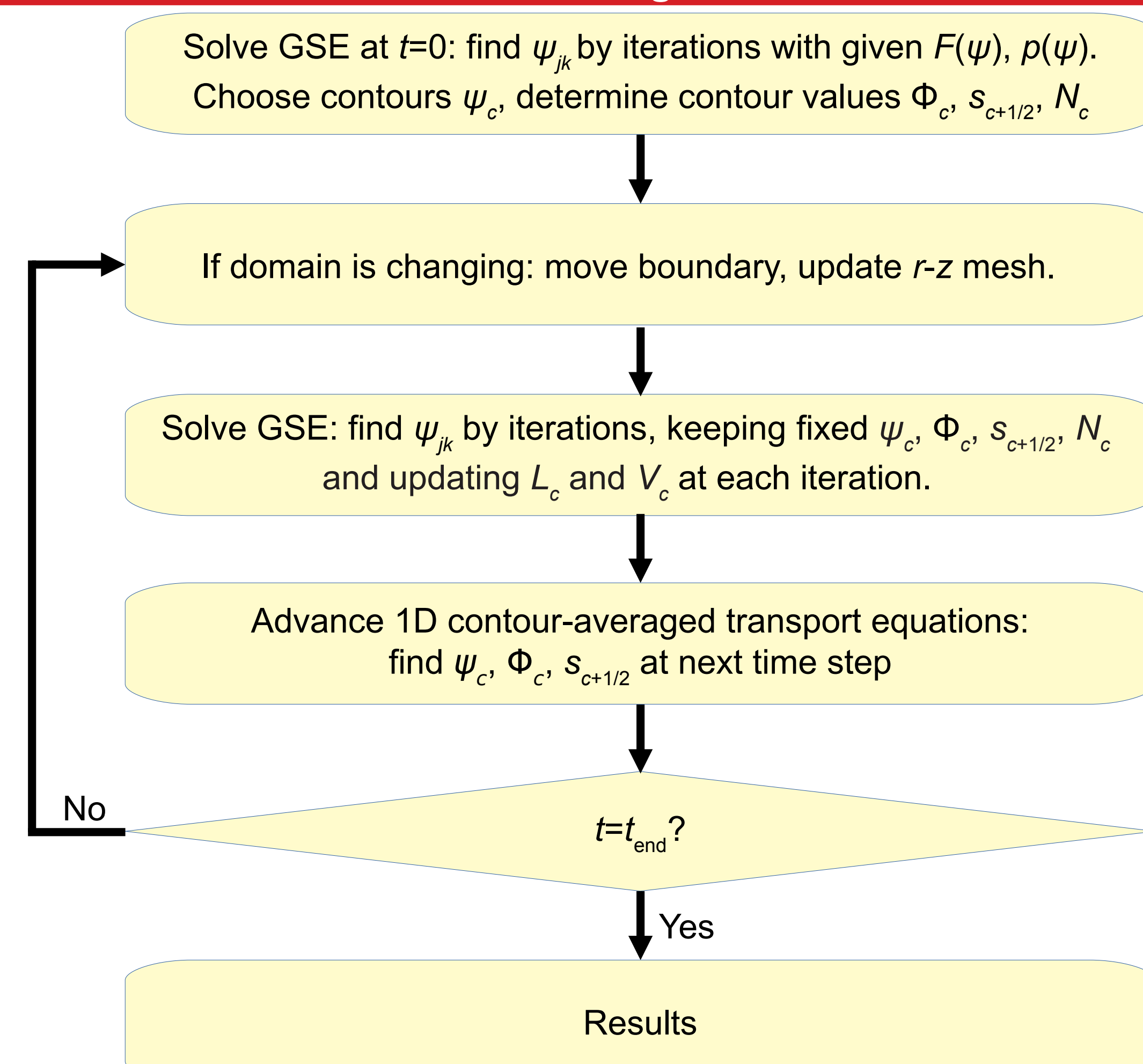
$$q_{c+\frac{1}{2}}^T + q_{c+\frac{1}{2}}^{\Omega} = \frac{d(Q_c^T + Q_c^{\Omega})}{dV} \Big|_{c+\frac{1}{2}} \approx \frac{(Q_{c+\frac{1}{2}}^T - Q_c^T) + \Delta Q_{c+\frac{1}{2}}^{\Omega}}{V_{c+1} - V_c}$$

$$Q_c^T = \int_c \frac{3}{2} \chi_{\perp} n k_B \nabla T \cdot d\mathbf{S} = \frac{3}{2} \chi_{\perp} n_c K_c k_B T'_{\psi} \Big|_c \approx \frac{3}{2} \chi_{\perp} n_c K_c k_B \frac{T_{c+\frac{1}{2}} - T_{c-\frac{1}{2}}}{\psi_{c+\frac{1}{2}} - \psi_{c-\frac{1}{2}}}$$

$$\Delta Q_{c+\frac{1}{2}}^{\Omega} \approx \frac{\pi}{\mu_0} (\eta_c (\Delta^* \psi)_c^2 + \eta_{c+1} (\Delta^* \psi)_{c+1}^2) (L_{c+1} - L_c) + \frac{\pi}{\mu_0} (\eta_c I_c F_{\psi}^2 \Big|_c + \eta_{c+1} I_{c+1} F_{\psi}^2 \Big|_{c+1}) (\psi_{c+1} - \psi_c)$$
- Additional integrals ( $-I_c \mu_0$  gives toroidal plasma current inside contour  $\psi_c$ ):
 
$$I_c = \frac{1}{2\pi} \int_c \frac{1}{r^2} \nabla \psi \cdot d\mathbf{S} = \int_{\psi(r,z) > \psi_c} \frac{\Delta^* \psi}{r} dr dz \approx \sum_{\psi_{jk} > \psi_c} \frac{[\Delta^* \psi]_{jk}}{r_{jk}} \Delta r_{jk} \Delta z_k$$

$$K_c = \int_c \nabla \psi \cdot d\mathbf{S} = \int_{\psi(r,z) > \psi_c} \Delta \psi 2\pi r dr dz \approx \sum_{\psi_{jk} > \psi_c} [\Delta \psi]_{jk} 2\pi r_{jk} \Delta r_{jk} \Delta z_k$$

## Numerical algorithm

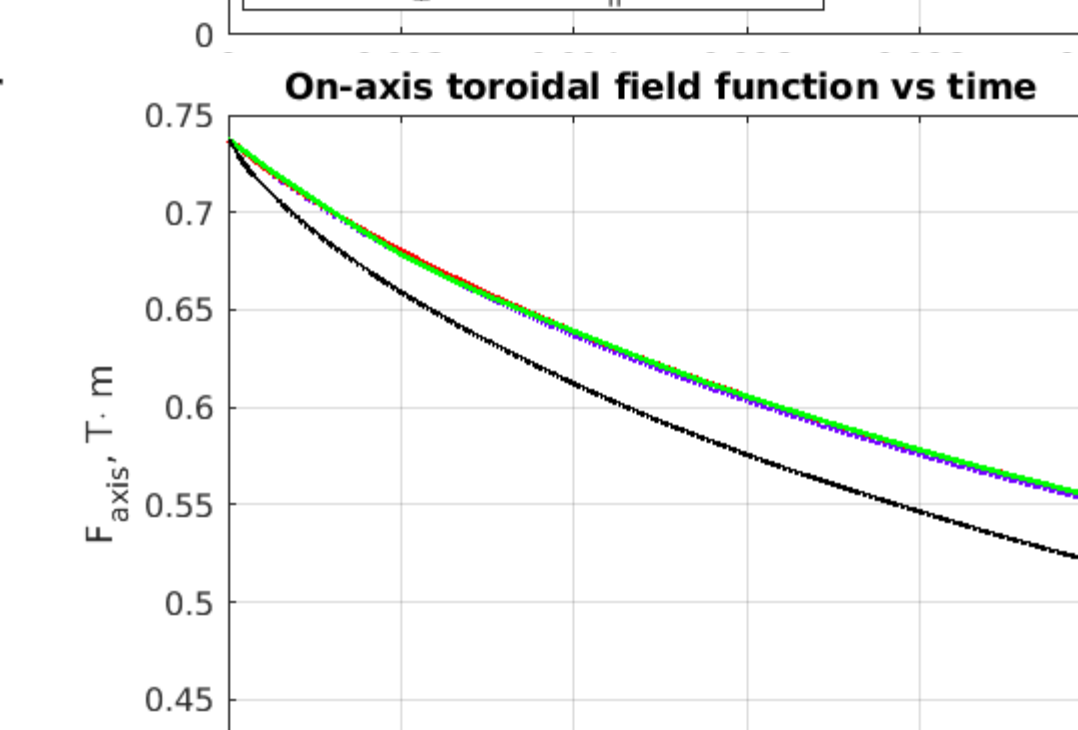
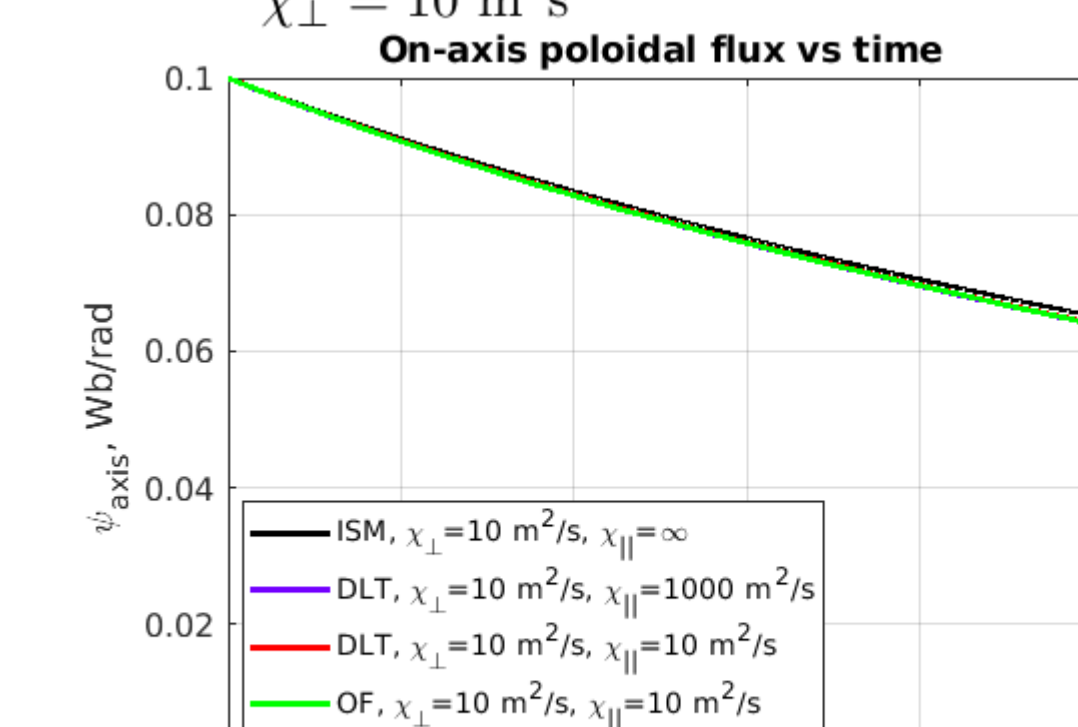
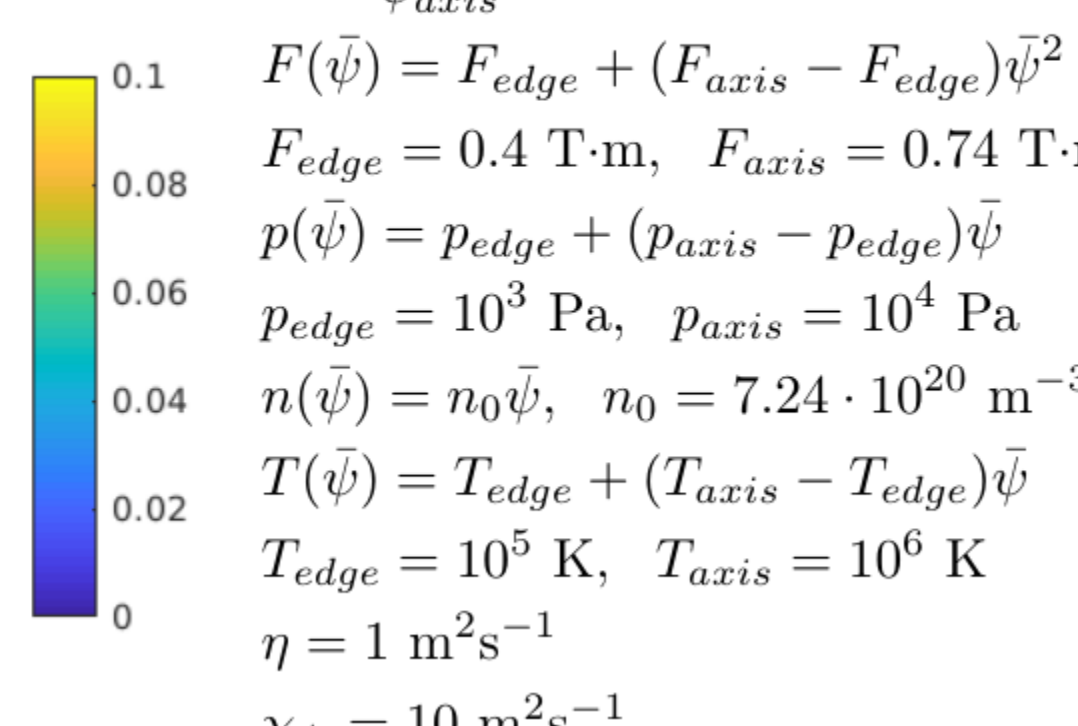
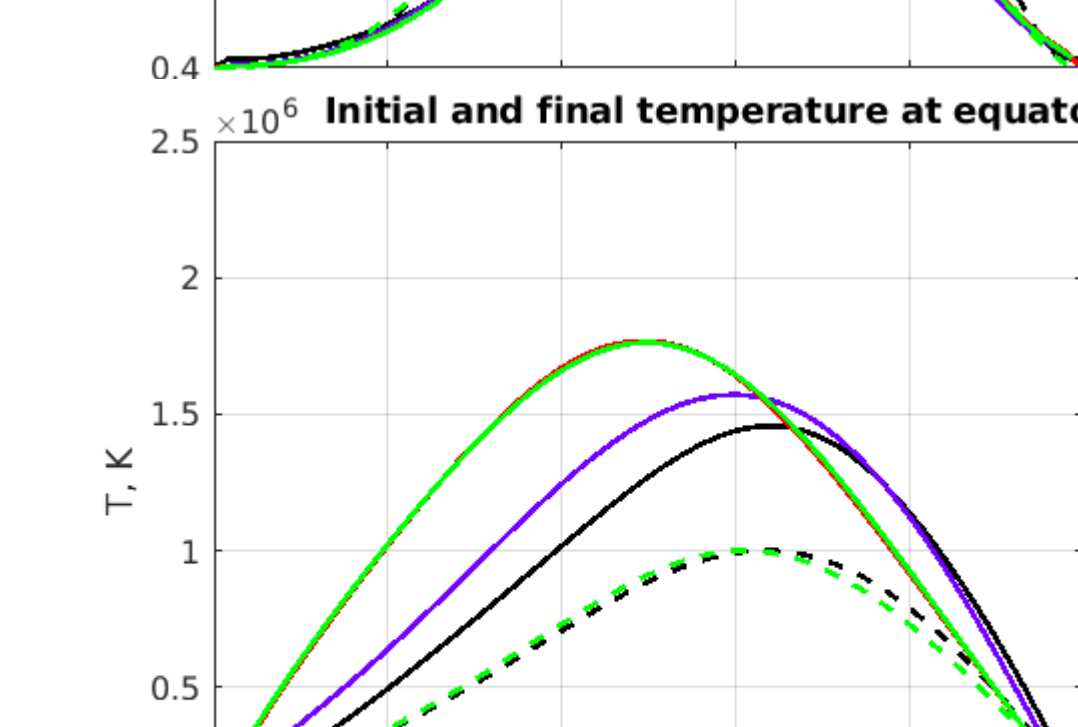
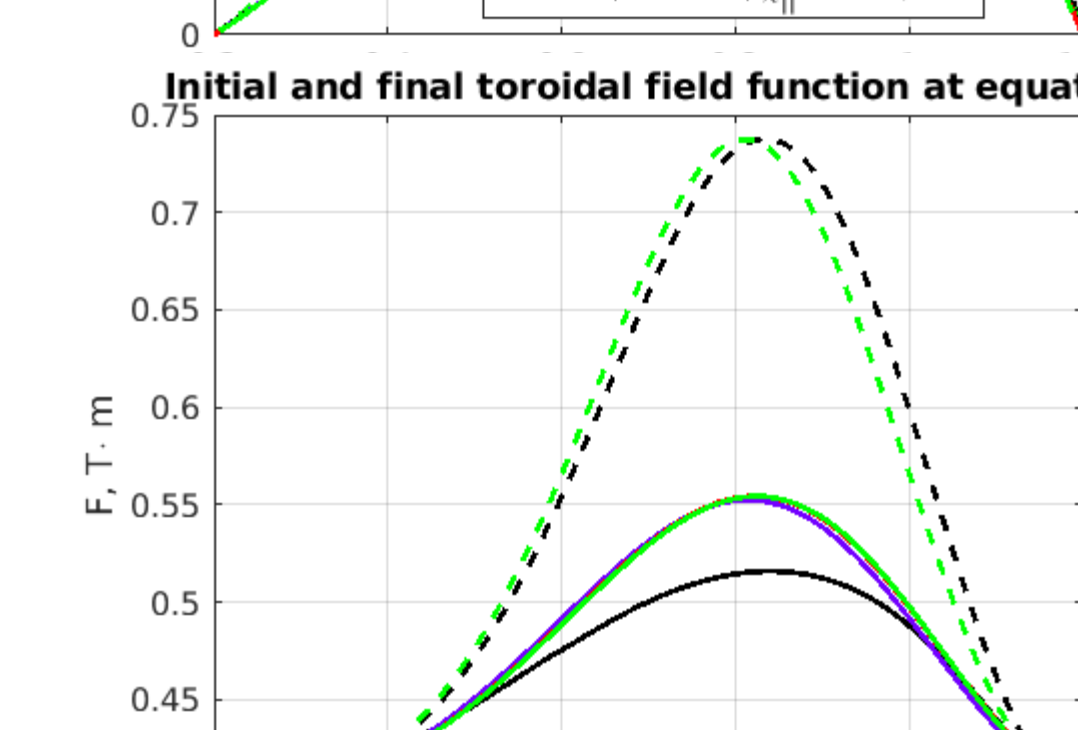
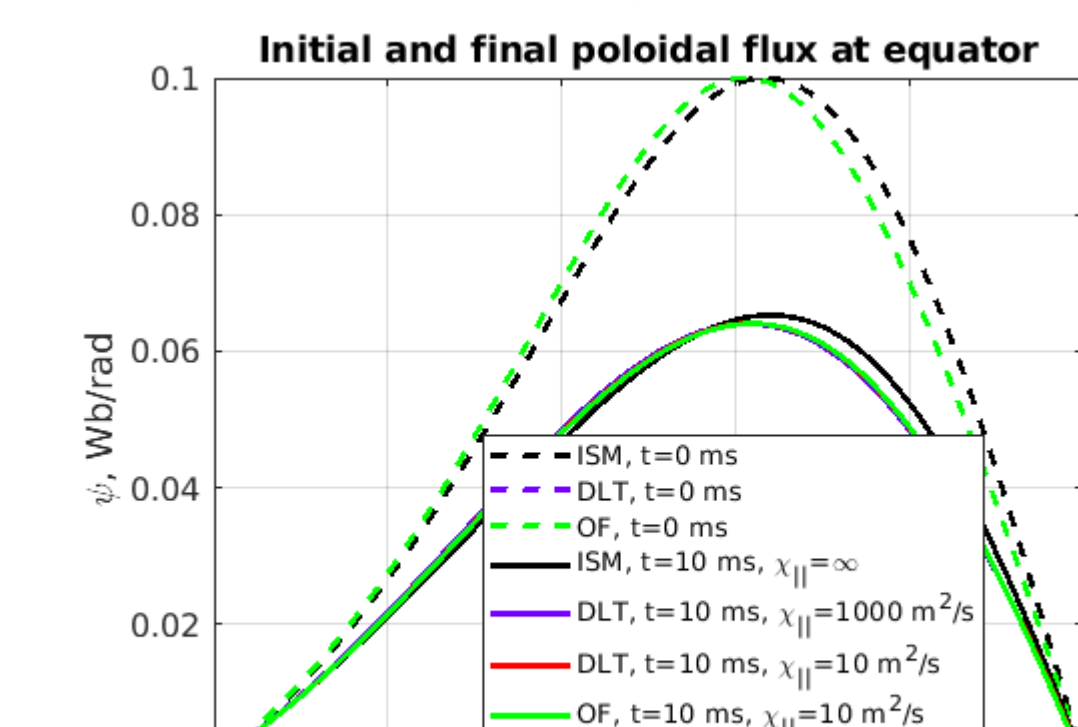
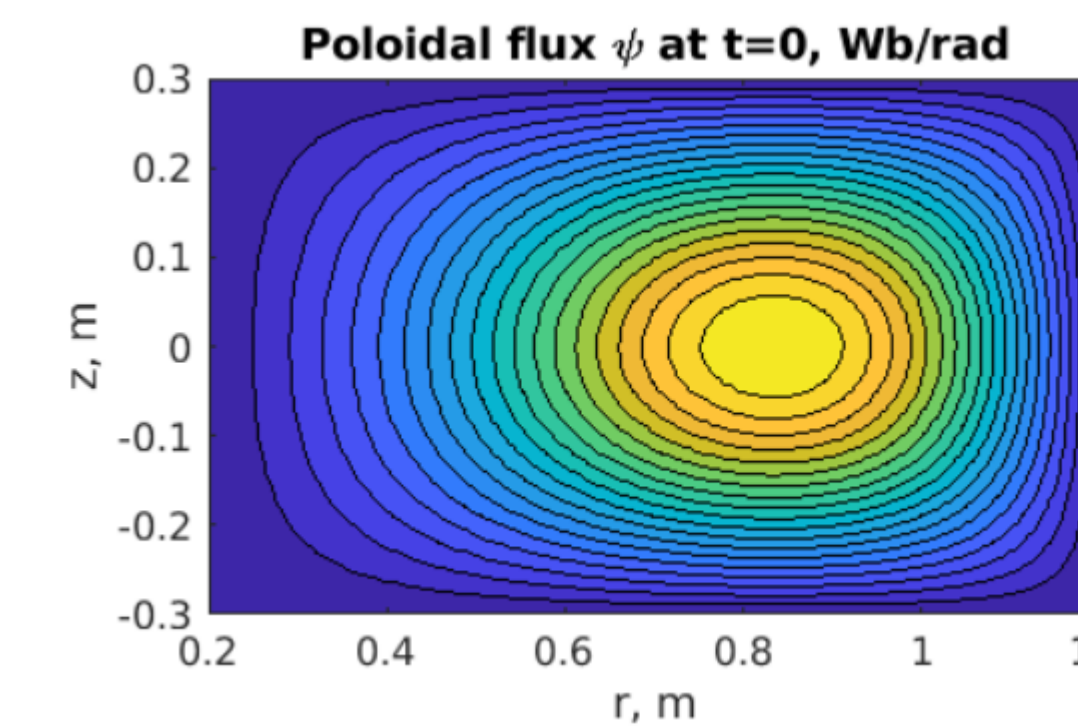


## STEADY GEOMETRY

- We perform comparison between ISM-plasma and two other MHD codes, DELITE [2] and OpenFOAM [3], for the test cases in steady geometry.

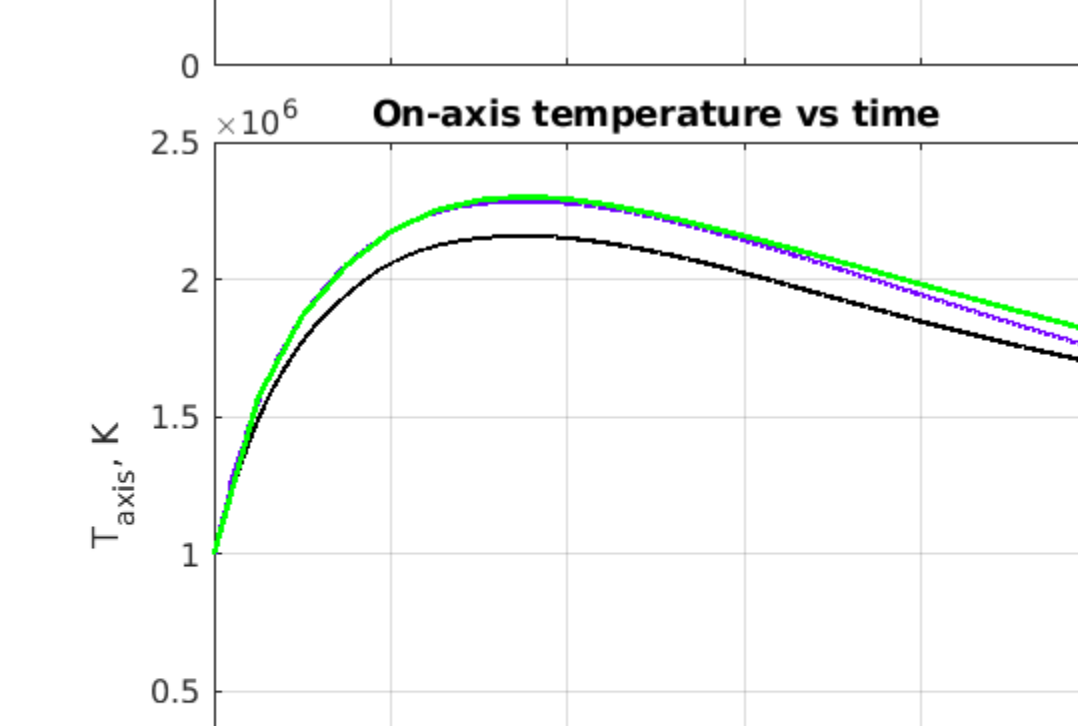
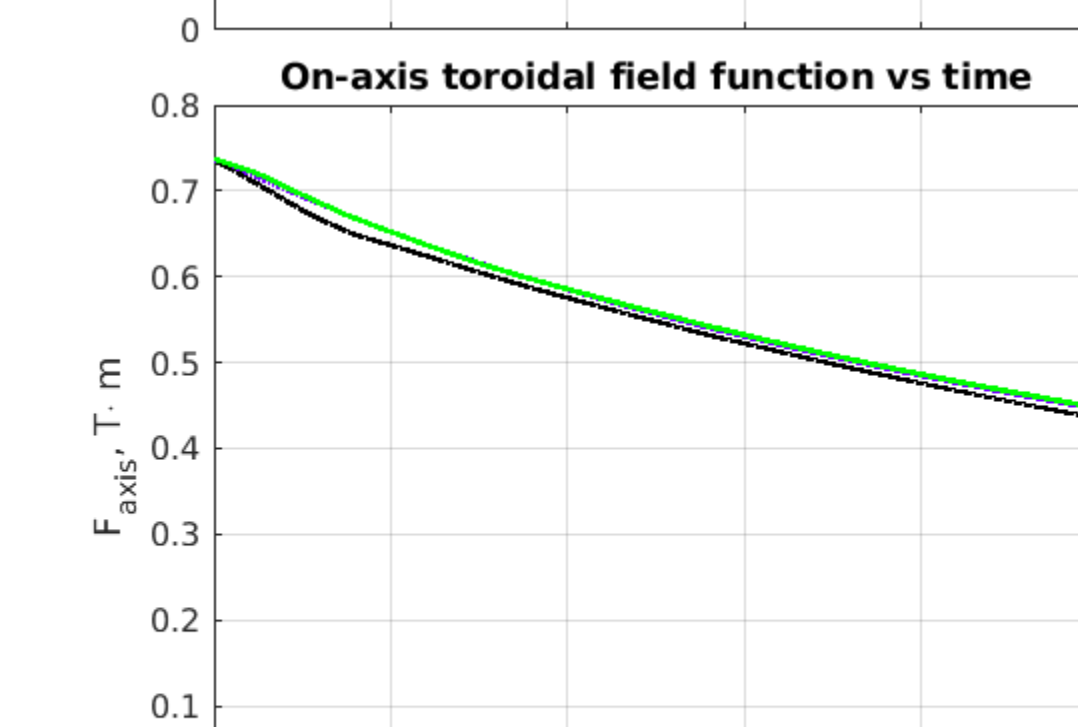
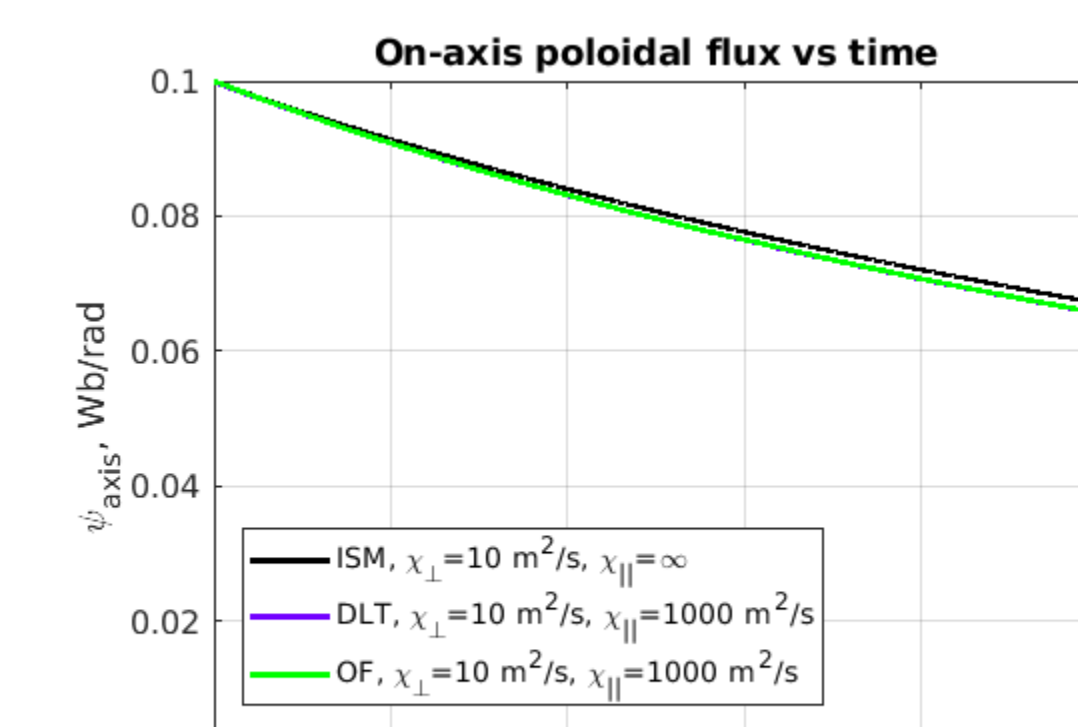
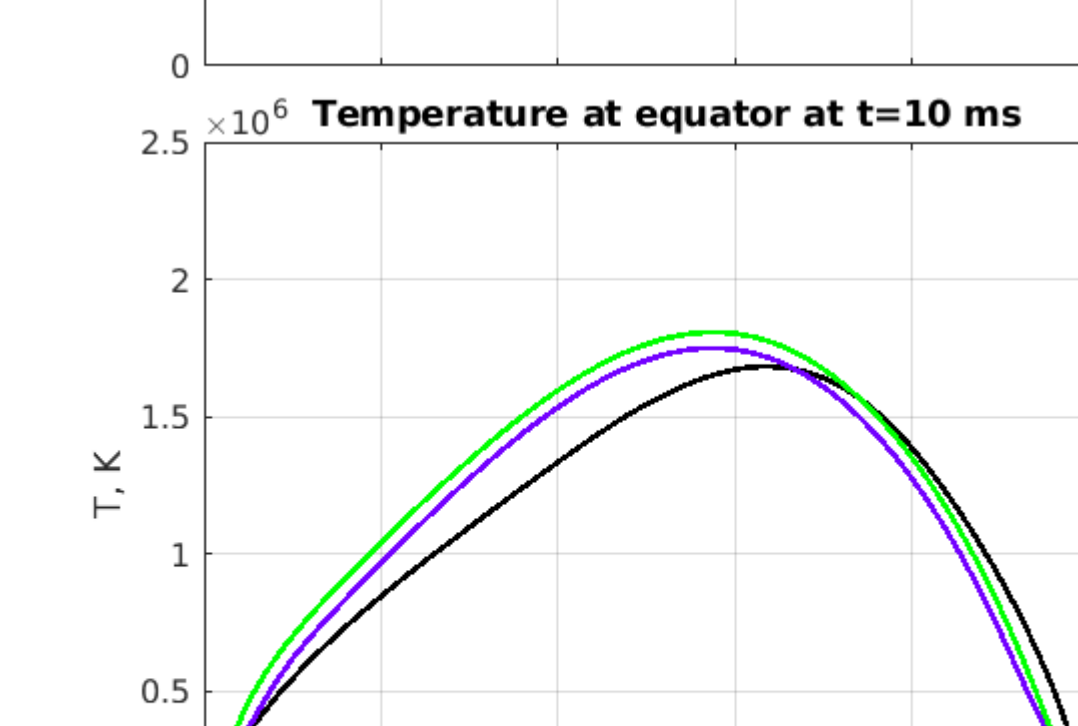
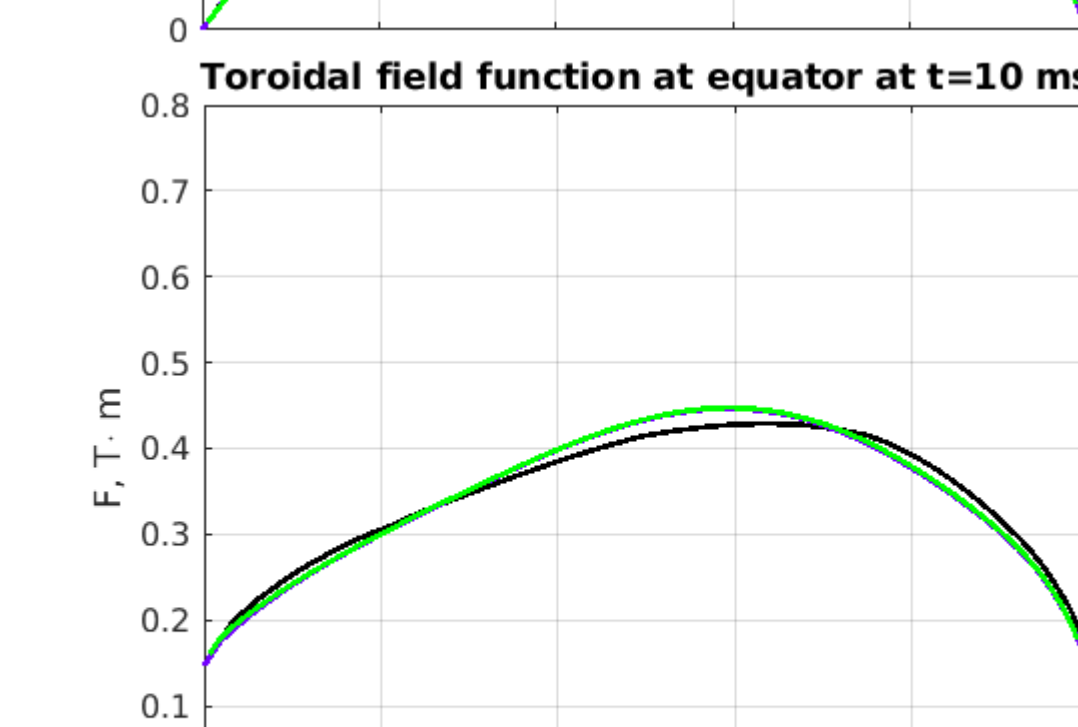
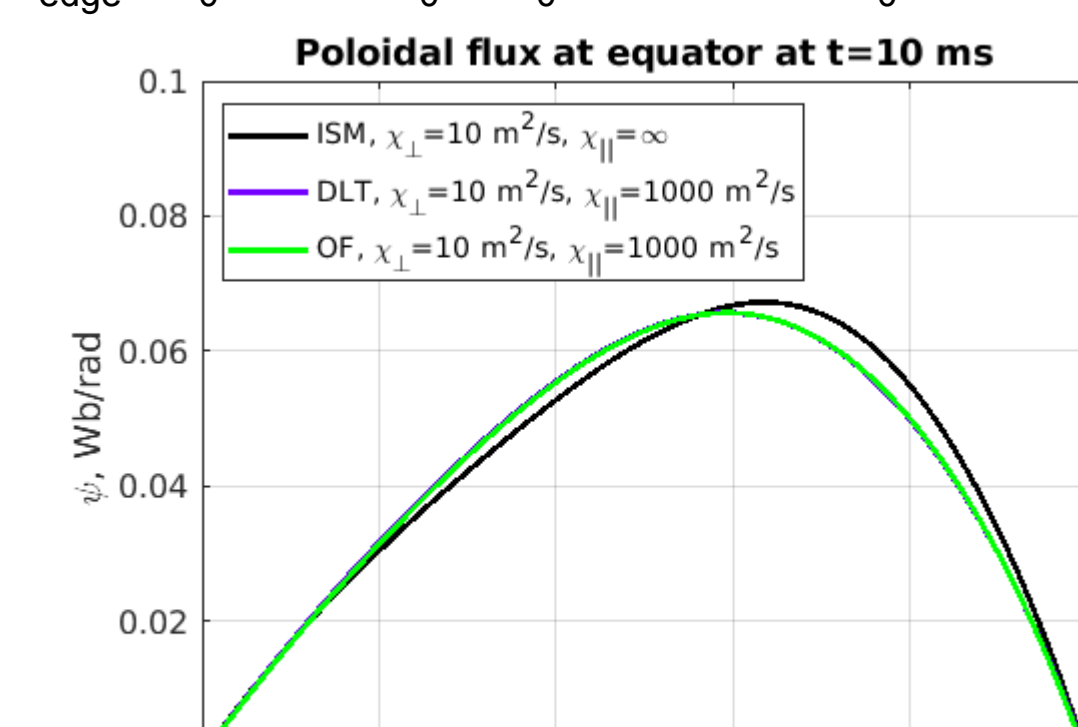
### Test case 1: constant shaft current

- Plasma is formed in a domain with rectangular cross section and resistively decays; toroidal field at the boundary  $F_{\text{edge}}$  (shaft current) is kept constant in time.
 **ISM-plasma numerical parameters:**  $N_r = 200, N_z = 60, N_c = 20, dt = 5 \cdot 10^{-6}$  s
 **Initial plasma parameters:**  $\tilde{\psi} = \frac{\psi}{\psi_{axis}}, \psi_{axis} = 0.1$  Wb/rad
  $F(\tilde{\psi}) = F_{\text{edge}} + (F_{\text{axis}} - F_{\text{edge}}) \tilde{\psi}^2$ 
 $F_{\text{edge}} = 0.4$  T·m,  $F_{\text{axis}} = 0.74$  T·m
  $p(\tilde{\psi}) = p_{\text{edge}} + (p_{\text{axis}} - p_{\text{edge}}) \tilde{\psi}^2$ 
 $p_{\text{edge}} = 10^3$  Pa,  $p_{\text{axis}} = 10^4$  Pa
  $n(\tilde{\psi}) = n_0 \tilde{\psi}, n_0 = 7.24 \cdot 10^{20}$  m $^{-3}$ 
 $T(\tilde{\psi}) = T_{\text{edge}} + (T_{\text{axis}} - T_{\text{edge}}) \tilde{\psi}^2$ 
 $T_{\text{edge}} = 10^5$  K,  $T_{\text{axis}} = 10^6$  K
  $\eta = 1$  m $^2$ s $^{-1}$ 
 $\chi_{\perp} = 10$  m $^2$ s $^{-1}$



### Test case 2: decaying shaft current

- The same as test case 1, but toroidal field at the boundary (shaft current) is decaying:  $F_{\text{edge}} = F_0 \exp(-t/t_0), F_0 = 0.4$  T·m,  $t_0 = 0.01$  s. This is relevant to the present GF experiment.



## COMPRESSION GEOMETRY

- In this case we demonstrate capability of ISM-plasma to model compression geometry.
- Plasma is formed in a shaped domain and then compressed by moving right boundary; the boundary trajectory is given as a function of time and mimics the motion of liner.
- Plasma and boundaries are perfect conductors.

### ISM-plasma numerical parameters:

$N_r = 100, N_z = 50, N_c = 20, dt = 2 \cdot 10^{-6}$  s

### Initial plasma parameters:

$\tilde{\psi} = \frac{\psi}{\psi_{axis}}, \psi_{axis} = 0.14$  Wb/rad

$F(\tilde{\psi}) = F_{\text{edge}} + (F_{\text{axis}} - F_{\text{edge}}) \tilde{\psi}^2$

$F_{\text{edge}} = 0.2$  T·m,  $F_{\text{axis}} = 0.25$  T·m

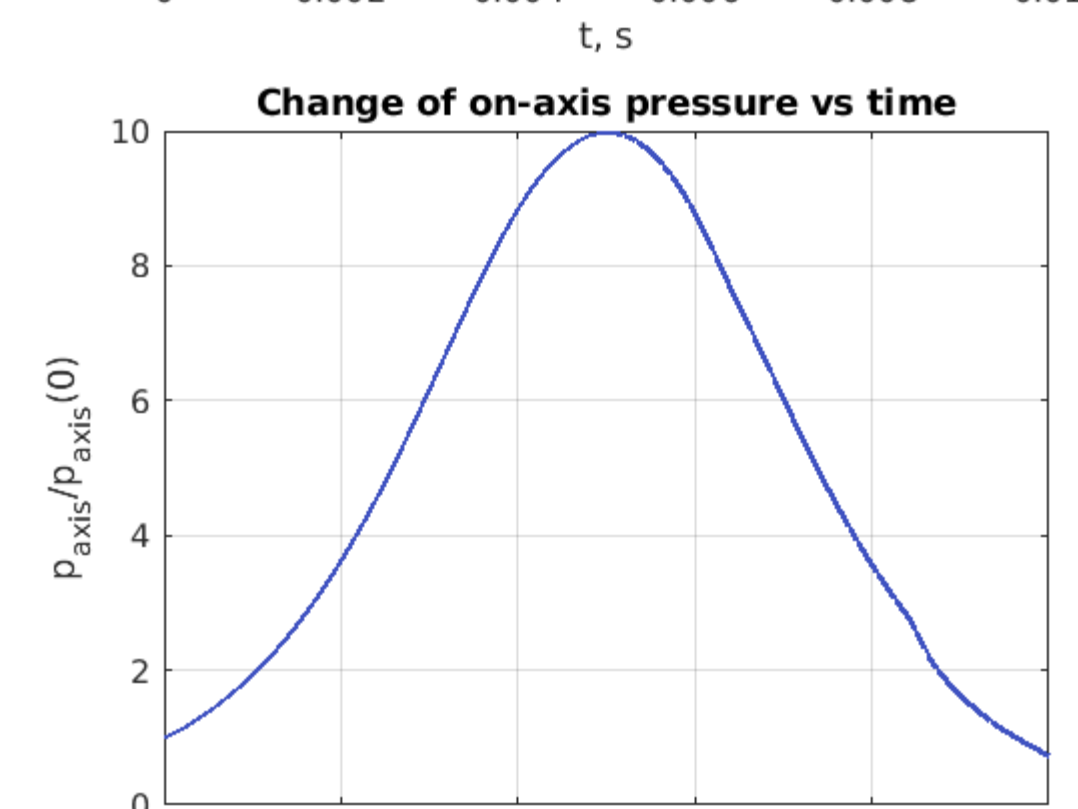
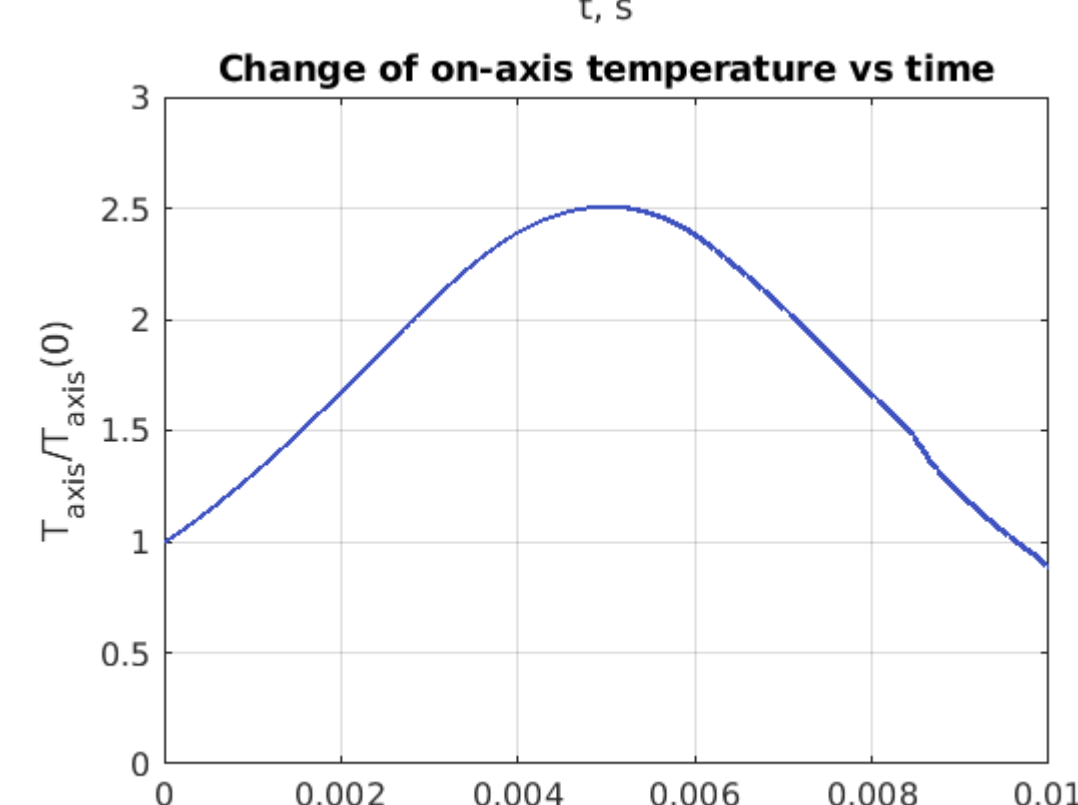
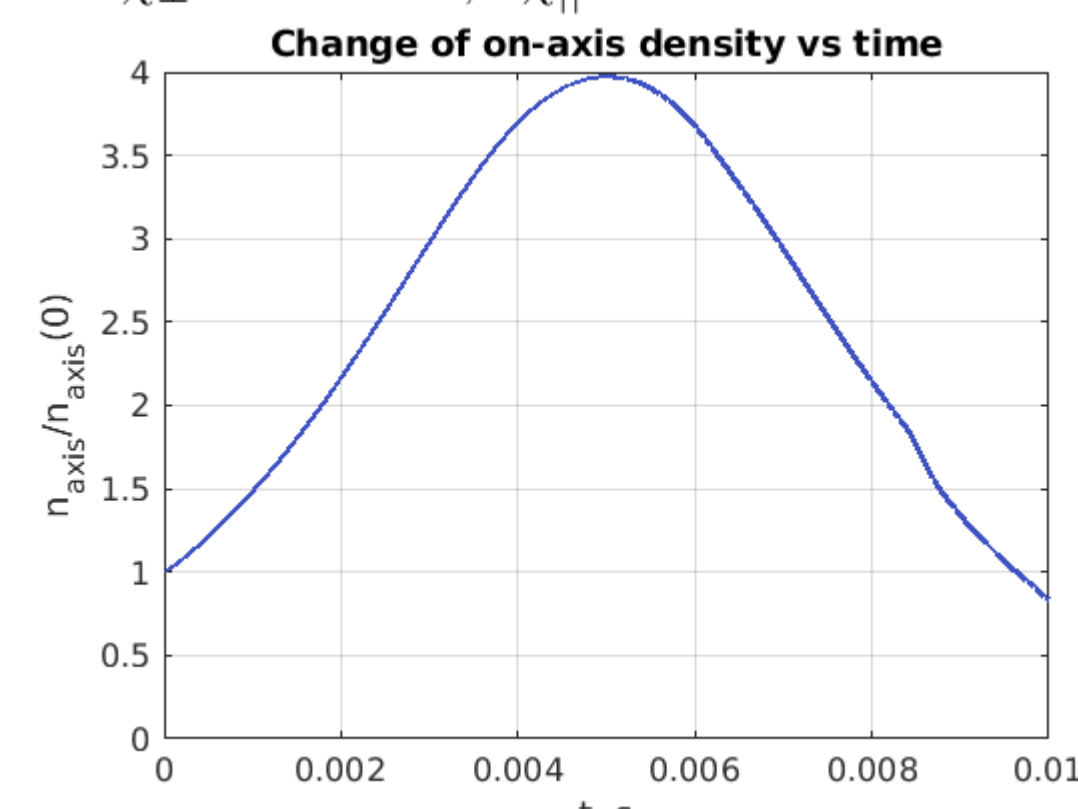
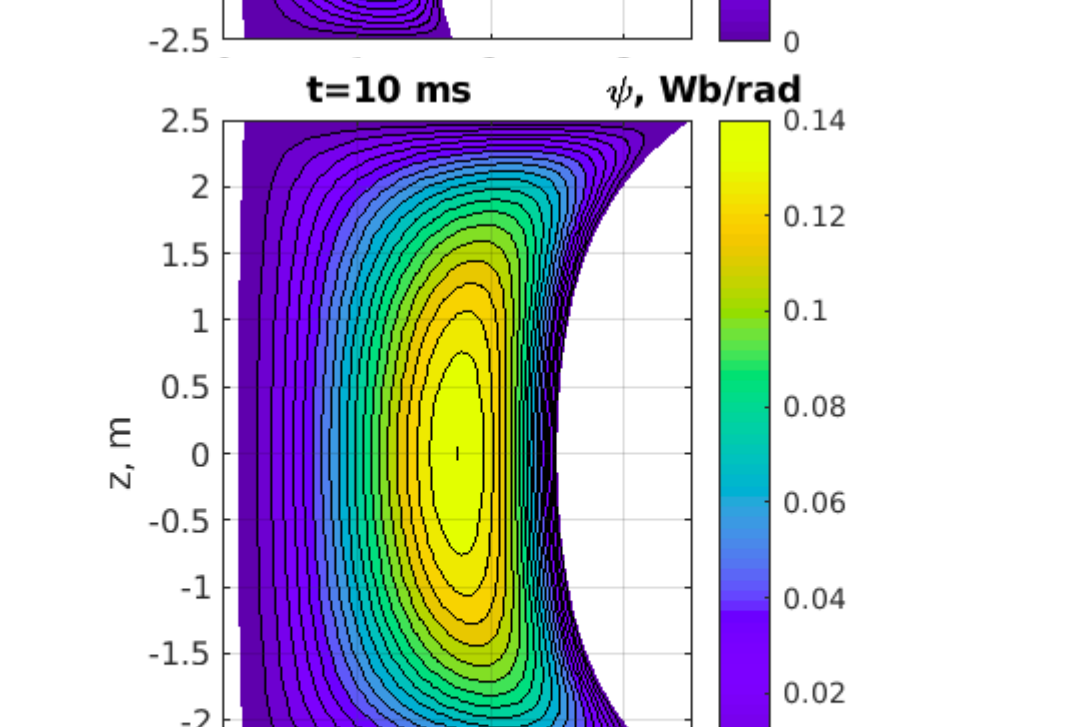
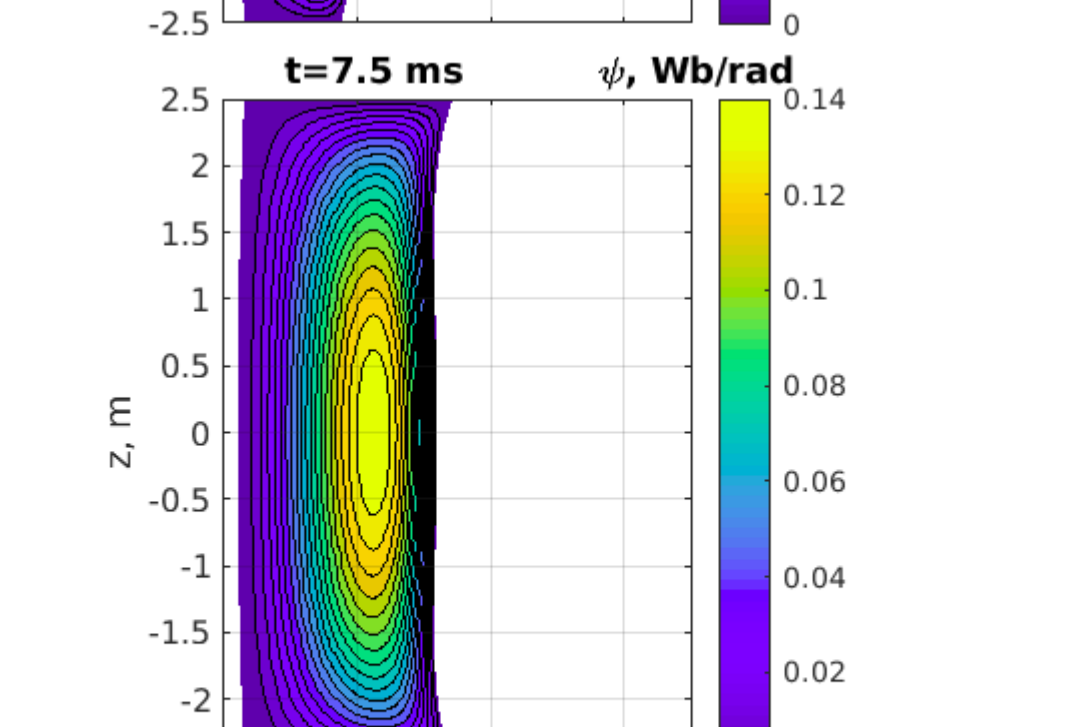
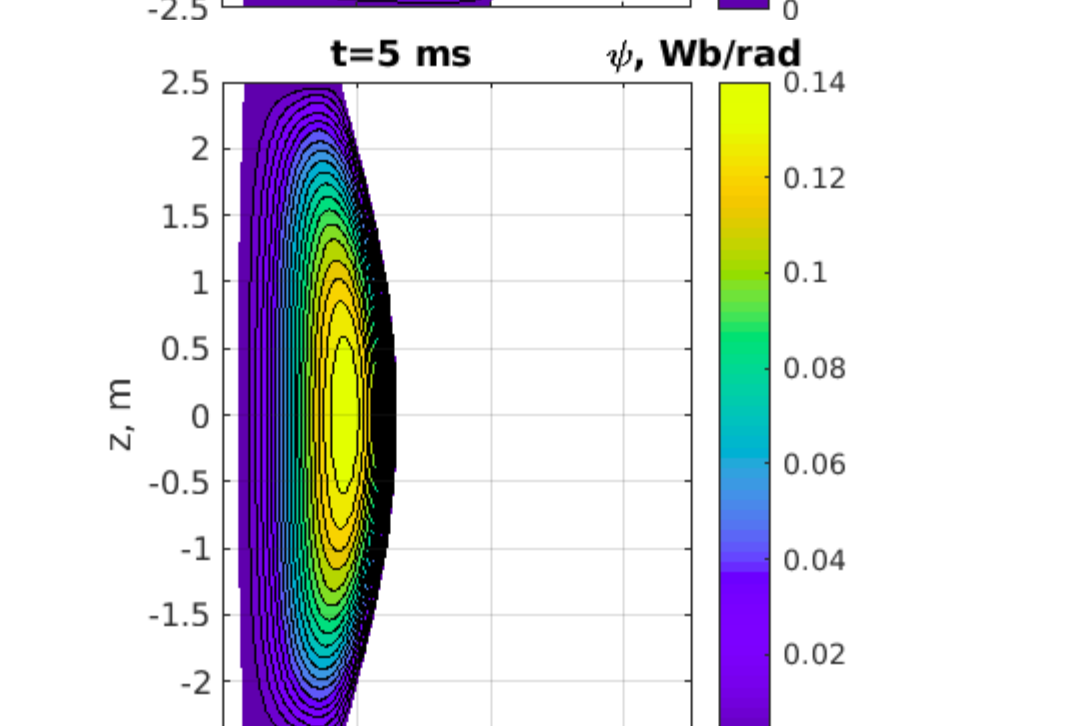
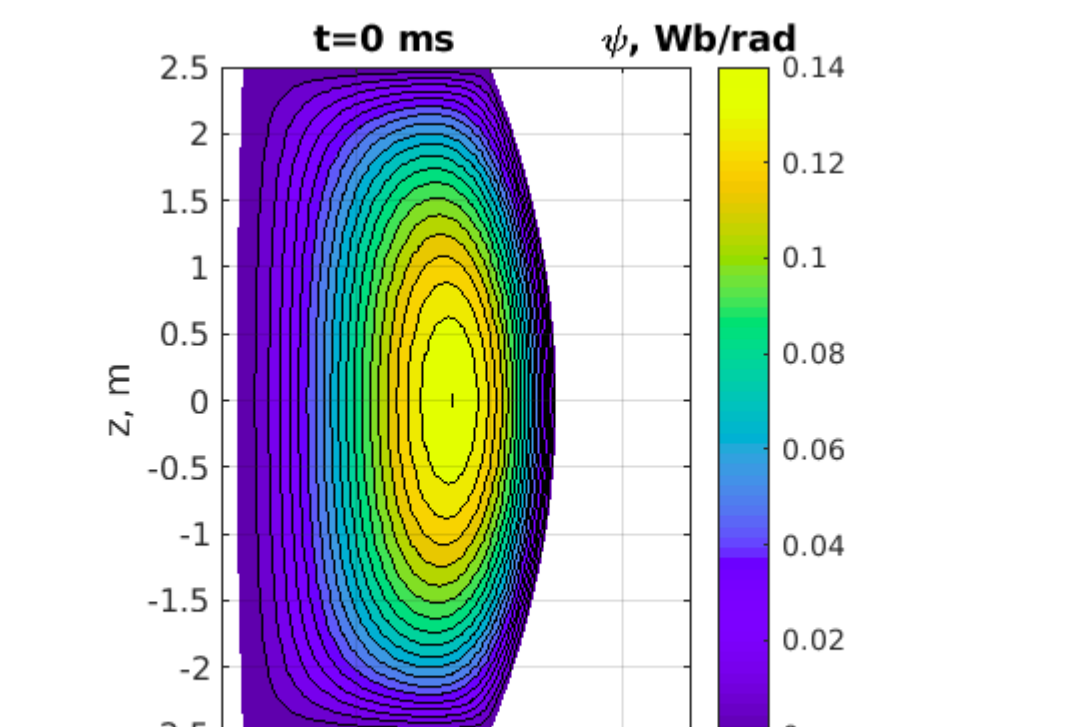
$p(\tilde{\psi}) = p_{\text{axis}} \tilde{\psi}, p_{\text{axis}} = 10^4$  Pa

$n(\tilde{\psi}) = n_0 \tilde{\psi}, n_0 = 7.24 \cdot 10^{20}$  m $^{-3}$

$T(\tilde{\psi}) = T_{\text{axis}} \tilde{\psi}, T_{\text{axis}} = 10^6$  K

$\eta = 0$  m $^2$ s $^{-1}$

$\chi_{\perp} = 0$  m $^2$ s $^{-1}, \chi_{\parallel} = \infty$



## CONCLUSIONS

- The ISM-plasma code is developed in General Fusion for simulating resistive evolution of plasma equilibrium in settings appropriate to GF MTF systems.
- The code is based on a contour method: at every time step it alternates between solving the 2D Grad-Shafranov equilibrium on Eulerian mesh and advancing the 1D transport equations discretized on a set of Lagrangian  $\psi$  contours.
- The main advantages of the code are:
  - speed (time step is limited by slow resistive timescale);
  - capability to model plasma inside domain with moving boundaries (as in MTF compression systems).
- Considered test cases show a good comparison of ISM-plasma with other MHD codes. The code performs very well in a steady geometry under conditions relevant to the present GF Plasma Injector 3 (PI3) experiment.
- Future development of the ISM-plasma code includes:
  - addition of vacuum poloidal field in plasma domain;
  - addition of field diffusion into liner;
  - coupling of ISM-plasma with self-consistent model for the liner motion determined from the ISM-hydro code [4].

## REFERENCES

- [1] H. Grad and J. Hogan "Classical Diffusion in a Tokamak", Phys. Rev. Lett. **24**, 1337 (1970).
- [2] C. Dunlea and I. V. Khalzov "A globally conservative finite element MHD code and its application to the study of compact torus formation, levitation and magnetic compression", arXiv:1907.13283 (2019).
- [3] V. Suponitsky, I. V. Khalzov and E. J. Avital "Magnetohydrodynamics solver for a two-phase free surface flow developed in OpenFOAM", Fluids **7**(7), 210 (2022).
- [4] I. V. Khalzov, D. Krotez, R. Ségas, "An interface tracking, finite volume code for modeling axisymmetric implosion of a rotating liquid metal liner with free surface", submitted to Computer Physics Communications (2023).